Illustrative examples of analysis and modeling of impurity erosion and redeposition experiments in DIII-D with integrated PMI models



#### **Outline**

- Modeling of W ring experiments in DIII-D:
  - C and W erosion/redeposition in DIII-D divertor can be consistently modeled using a Monte-Carlo impurity transport code and sheath & material reduced models\*:
    - → Experimental and theoretical framework in DIII-D to validate and use impurity transport code in Tokamak conditions (GITR)
  - Accurate modeling of C deposition on W may however require a more detailed material model:
    - → Experimental framework in DIII-D to validate integrated models of surface evolution and roughness, material erosion and impurity transport
- Modeling W redeposition with ion-gyro sheath:
  - reduced model vs PIC model?
    - → Example of experimental framework in DIII-D to benchmark PIC simulations with ITER relevant physics

<sup>\*</sup> Presented at the 23<sup>rd</sup> international conference on Plasma Surface Interactions in Controlled Fusion Devices

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#### Modeling of C and W erosion/redeposition in DIII-D divertor

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- Why modeling W net erosion is challenging?
- Measurement of W gross erosion and outboard deposition in DIII-D lower divertor with a toroidally symmetric W source

- Modeling and analysis of W gross erosion mechanism
  - W sputtering results from synergetic effects between impurity erosion, implantation, redeposition and transport processes
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In-situ exp. measurements of W gross erosion

WI (400.9 nm) +SXB

[AbramsNF2016,DingNF2017,HakolaPS2016]

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Post-mortem analysis of W deposition on PFCs [RudakovPS2014]

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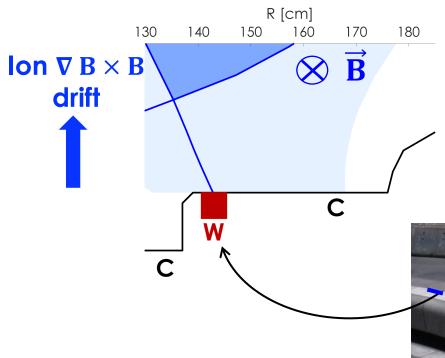
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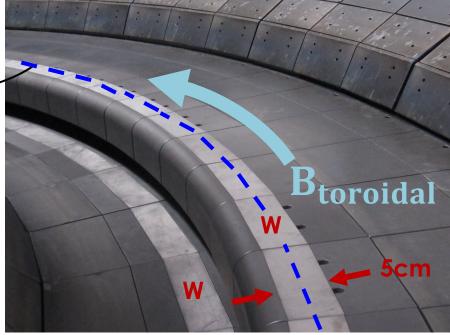
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Can dedicated experiments with <u>localized toroidally symmetric W source</u> in divertor improve understanding of mechanisms governing W net erosion and transport?

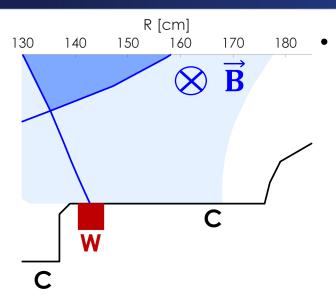
## W metal ring experiments in DIII-D: introducing a localized and toroidally symmetric W source in the DIII-D lower divertor



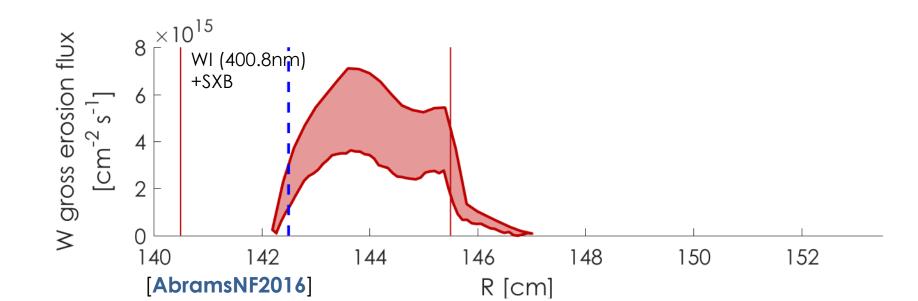
- W rings in DIII-D lower outer divertor:
  - localized and toroidally symmetric W source
- 25 repeated attached L-mode shots in reverse Bt-field with outer strike point on the outboard W ring



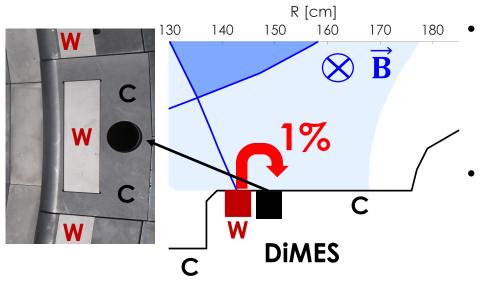
#### $\Gamma_{\rm W}^{\rm ero}$ ~0.1% $\Gamma_{\rm D}$ on W ring and net deposition of C on W near the separatrix



In-situ measurement of W gross erosion  $\sim 0.1\%$   $\Gamma_D$ , comparable to fraction in experiments with localized W source [DingNF2016]



#### Outboard W deposition $\sim 1\% \Gamma_{W}^{ero}$ measured at 3.5cm from W outer edge

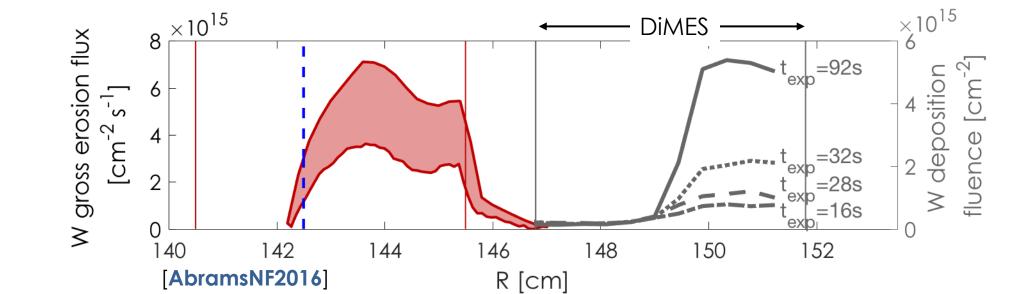


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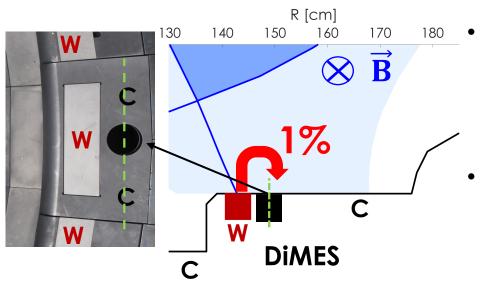
Inter-shots measurements of W outboard deposition

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$$\Gamma_W^{dep} \sim 5 \times 10^{13} \text{ cm}^{-2} \text{ s}^{-1} \sim 1\% \Gamma_W^{ero}$$

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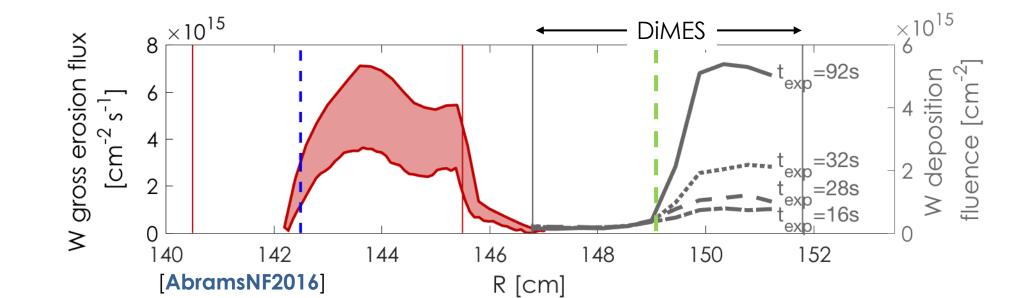


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very localized W deposition at 3.5cm from W outer edge



#### Modeling of C and W erosion/redeposition in DIII-D divertor

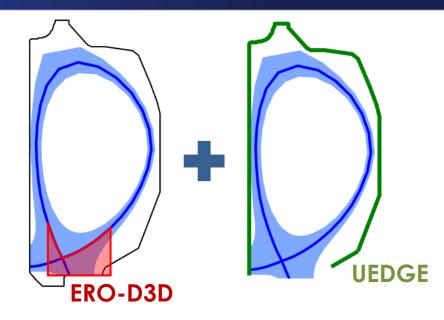
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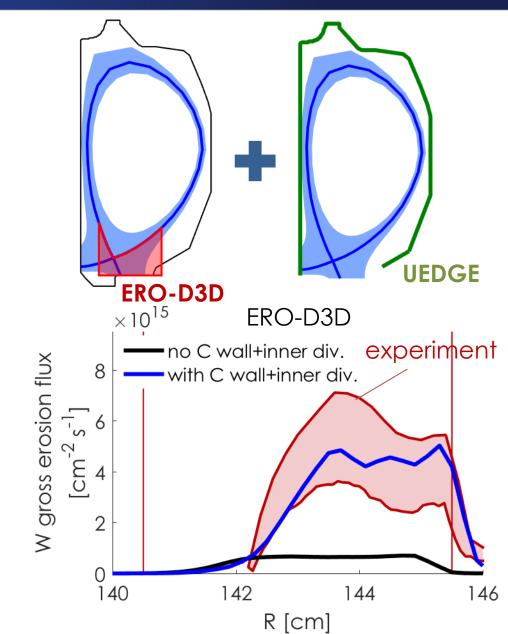
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  - 3D + ion gyro-sheath + collisions with D + ExB & ∇B × B drifts
  - C+W homogenous mixed-material model [KriegerJNM1993]
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- Carbon source from main chamber wall + inner divertor calculated with UEDGE (2D fluid code) [Rognlien2000]



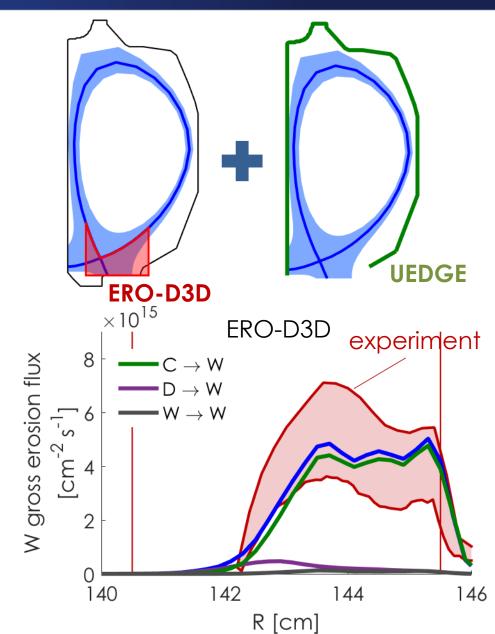
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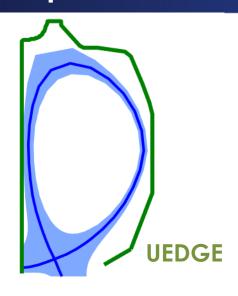


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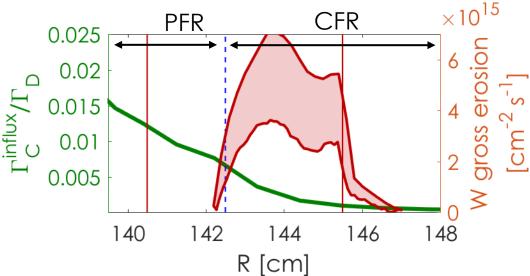
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### Influx of C from inner divertor and main chamber wall on W is localized near the separatrix

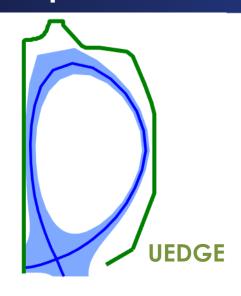


C influx at the outer divertor target without C source from outer divertor

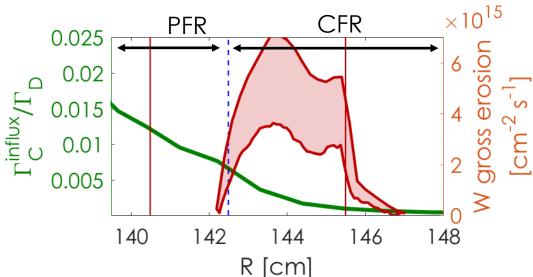


- Small C influx on W in the common flux region (CFR): ~0.1%  $\Gamma_{\! D}$
- Large C influx on W near the separatrix:  $\sim 1\% \Gamma_{\rm D}$

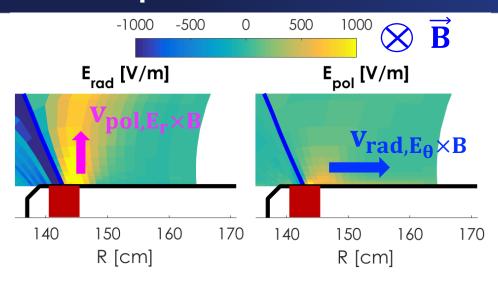
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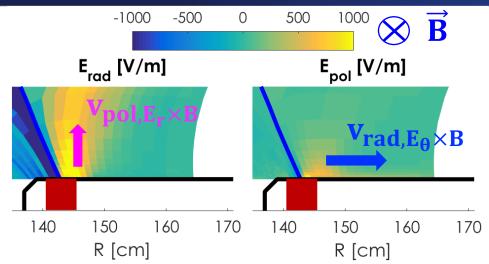




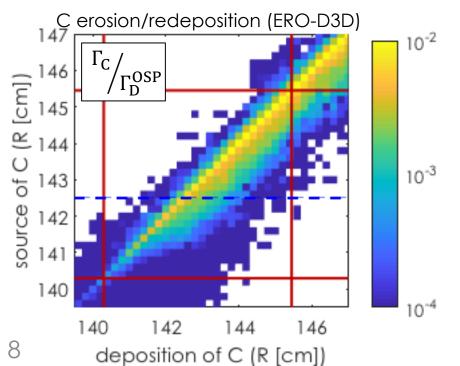
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- W gross erosion mainly occurs in the common flux region: how C migrate on W from the separatrix into the common flux region?

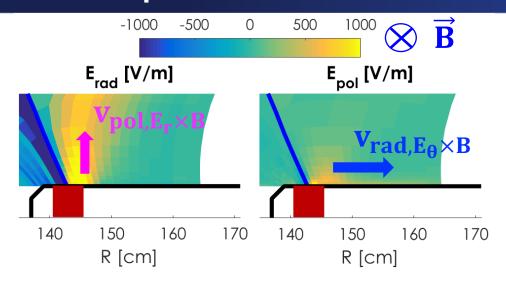


- Downward poloidal ExB drift in PFR/Upward poloidal ExB drift in the CFR
- Outward radial ExB drift in the CFR

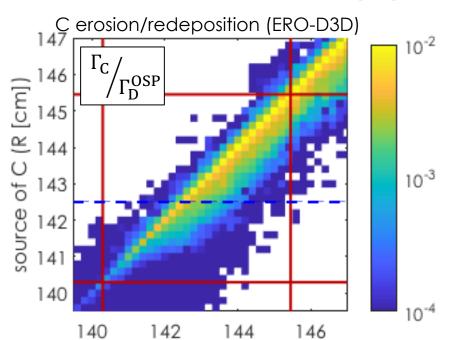


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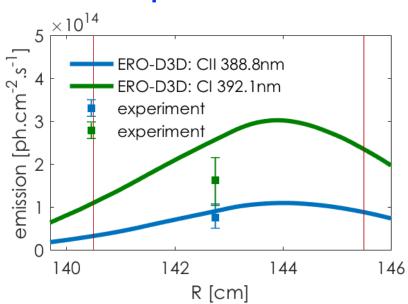


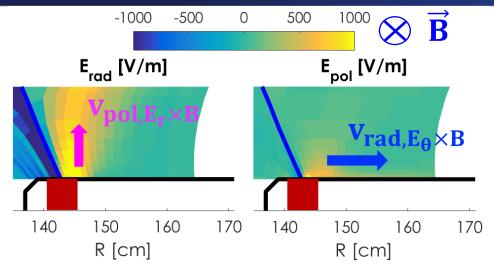
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deposition of C (R [cm])

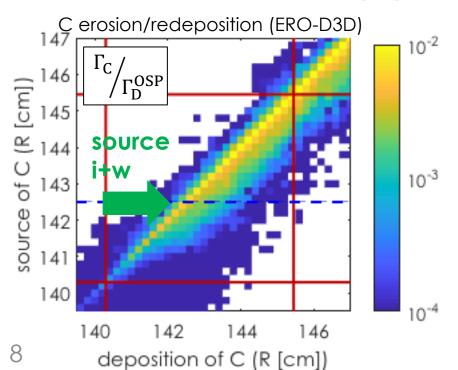
 C content above W predicted with ERO-D3D in agreement with experimental observations

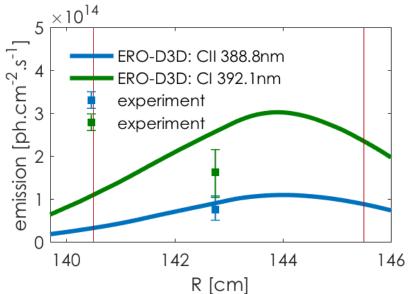


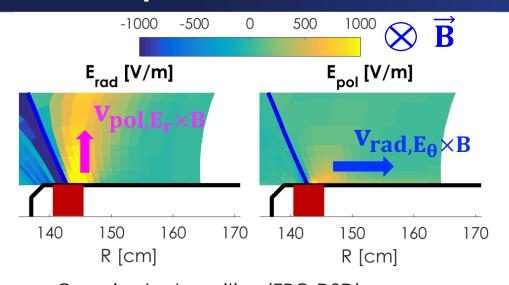


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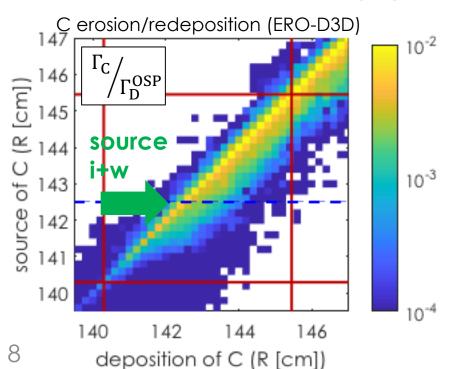


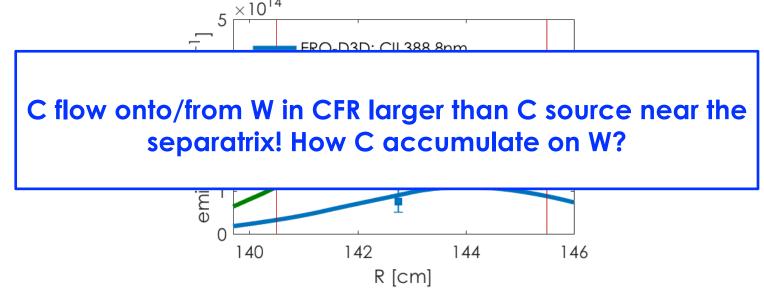






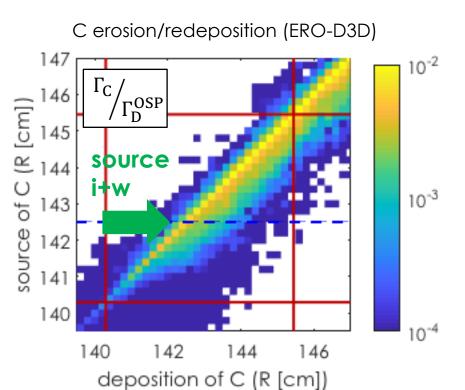
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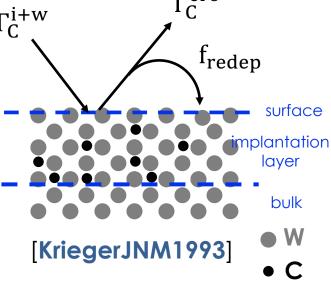




• C implantation in W described by the homogenous mixed material model in ERO-D3D  $_{\Gamma^{i+w}}$   $_{\Gamma^{er}_{C}}$ 

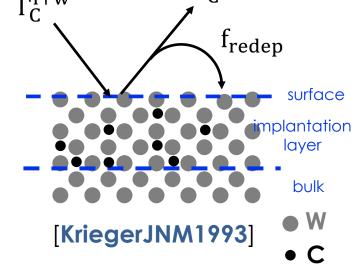
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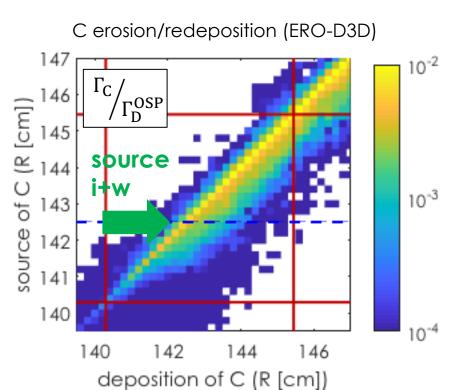


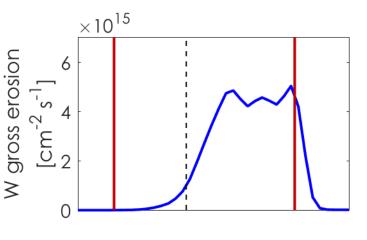


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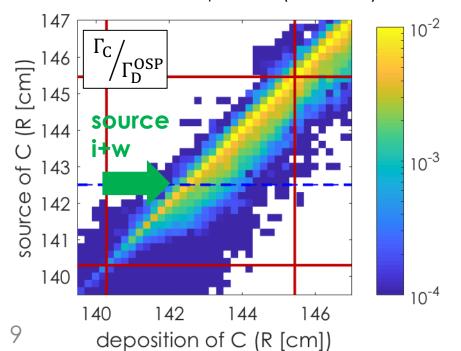
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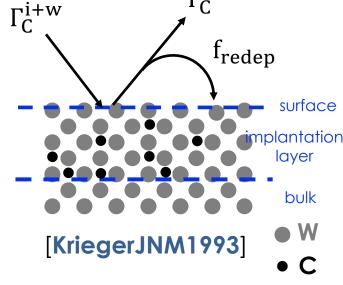


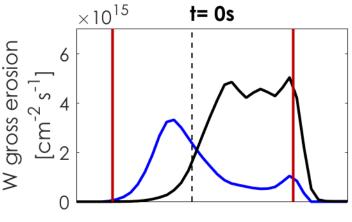
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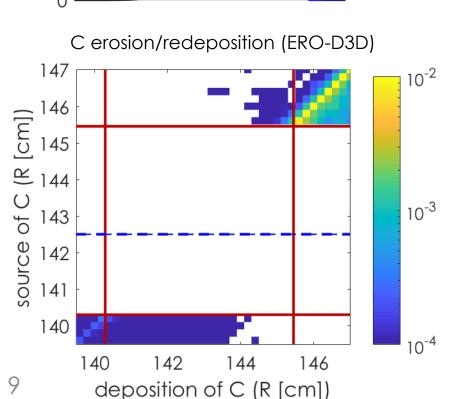


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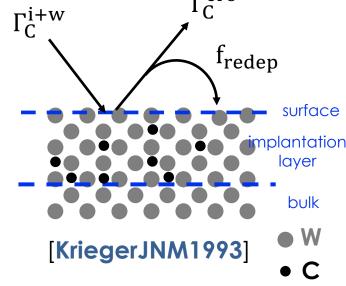


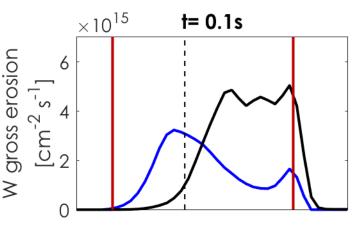




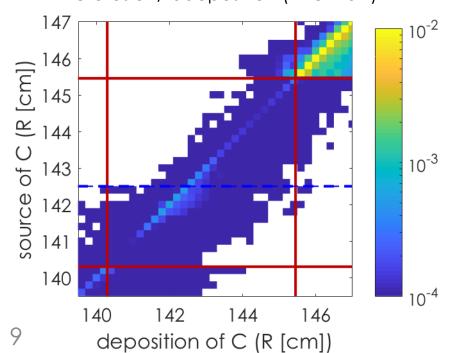
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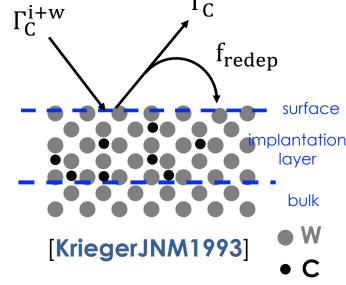


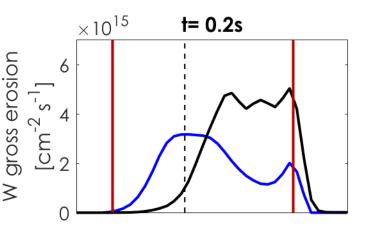
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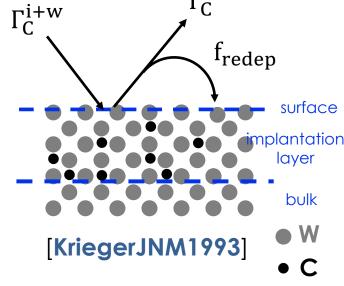


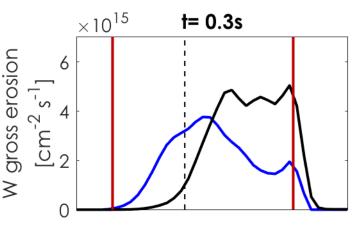


deposition of C (R [cm])

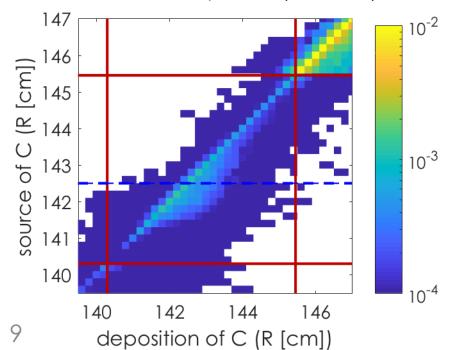
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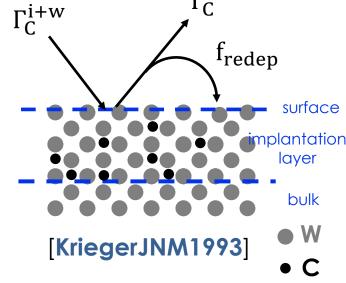


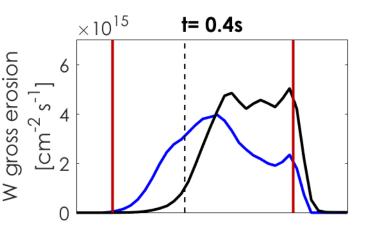
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C erosion/redeposition (ERO-D3D)

147
146

145
2 144

O 143

0 142

0 141

140

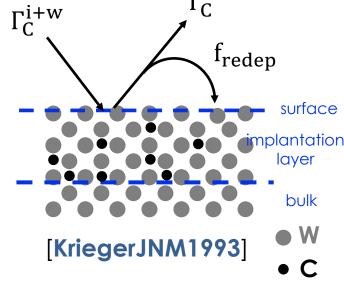
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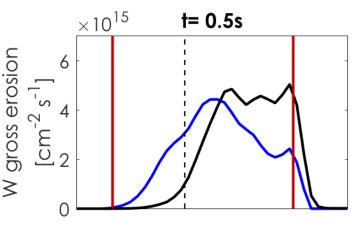
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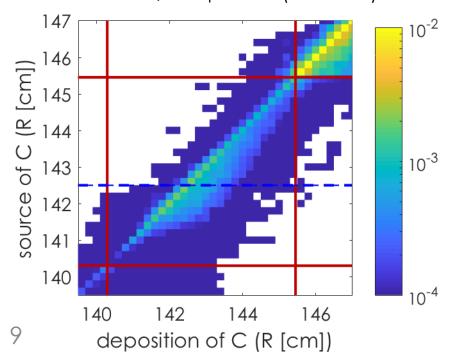
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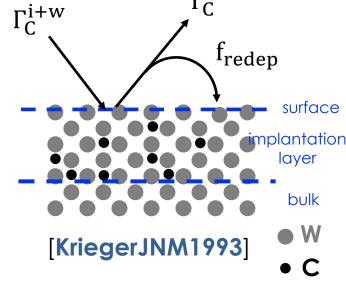


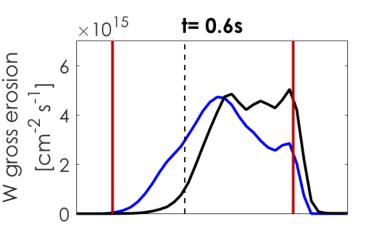
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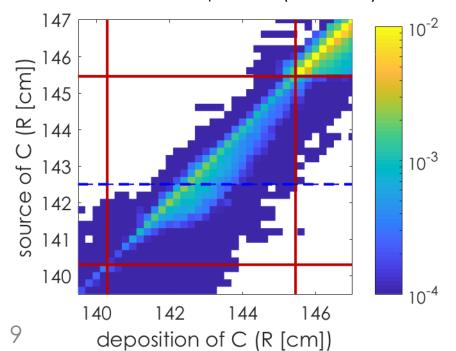
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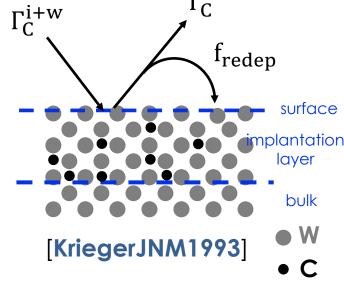


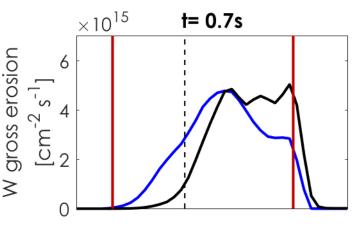
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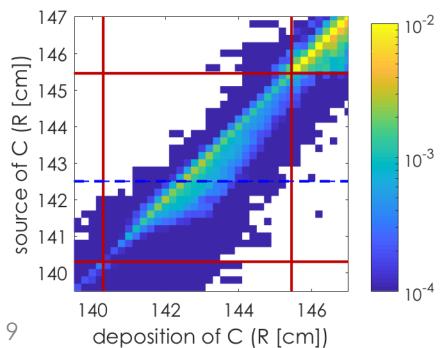
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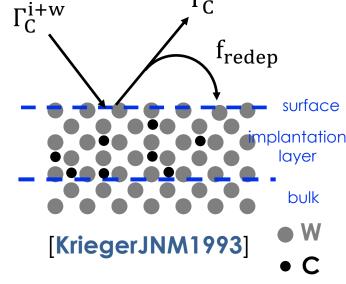


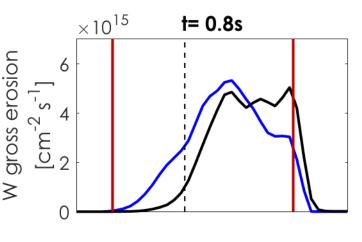
C erosion/redeposition (ERO-D3D)



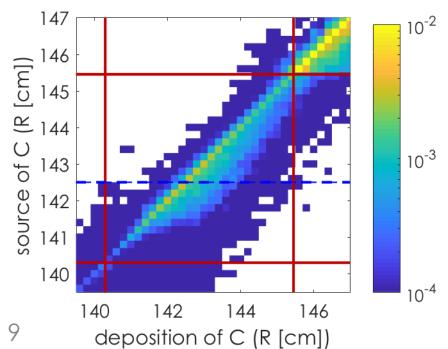
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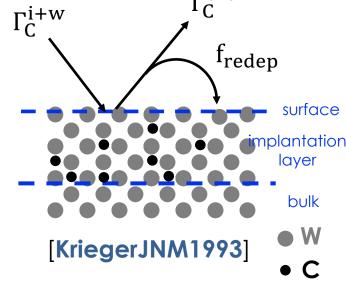




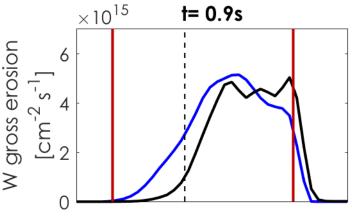


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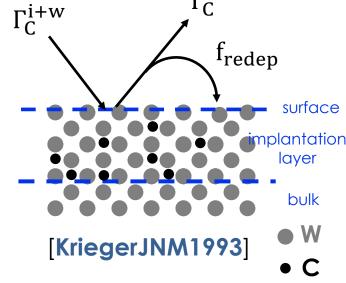
Implantation of C in W and large C redeposition onto W induces large C flux on W (C "recycling")



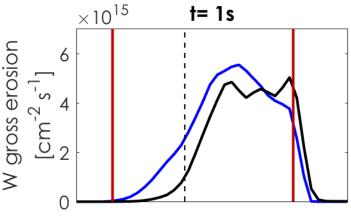
C erosion/redeposition (ERO-D3D) 147 146 [cm]) 145 144  $10^{-3}$ 143 source 42 142 146 deposition of C (R [cm])

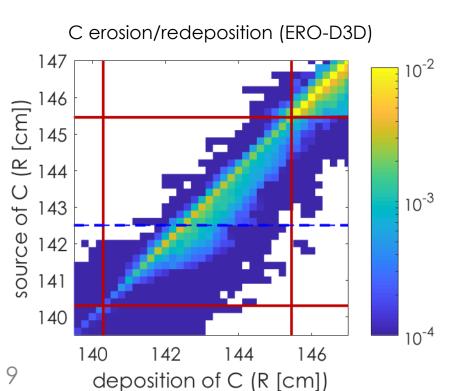
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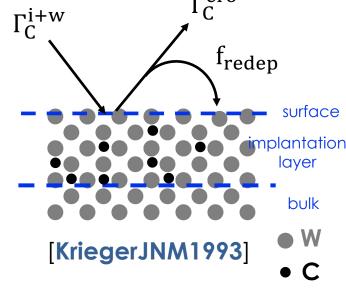
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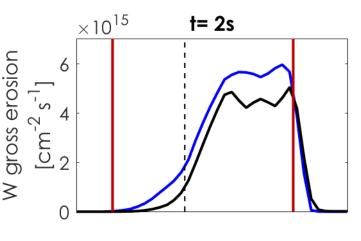


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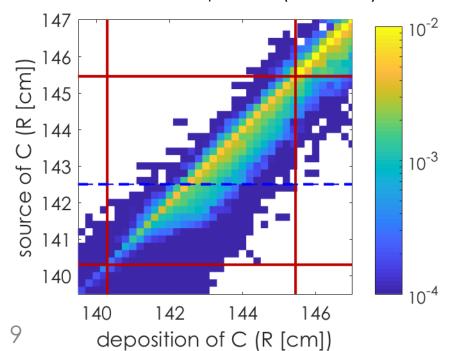
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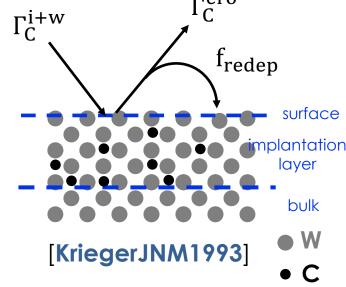




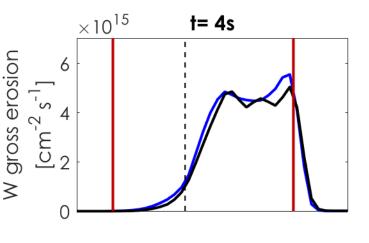


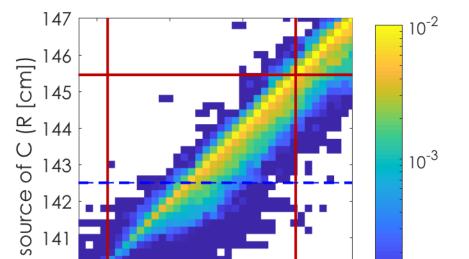
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146

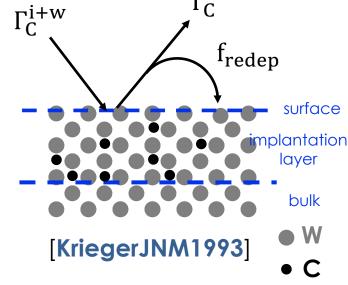
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deposition of C (R [cm])

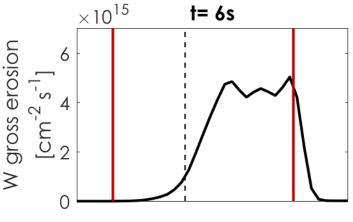
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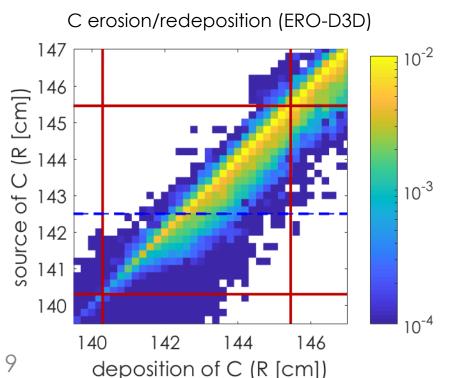
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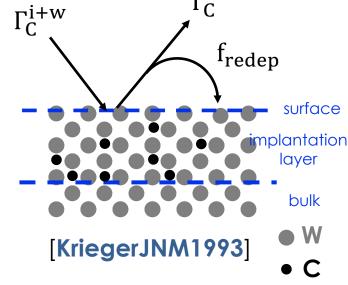
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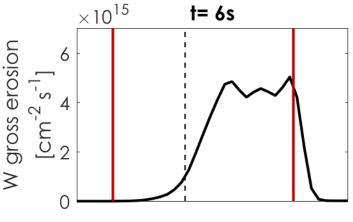


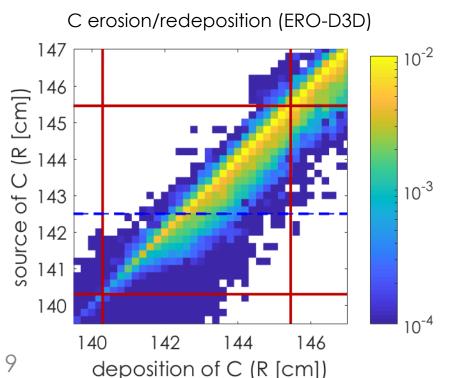
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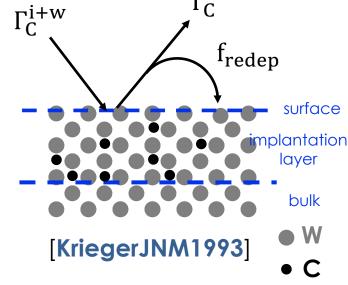
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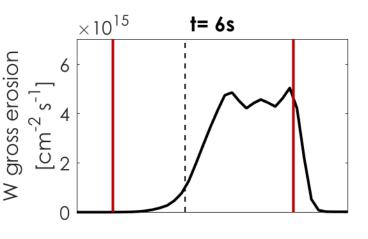


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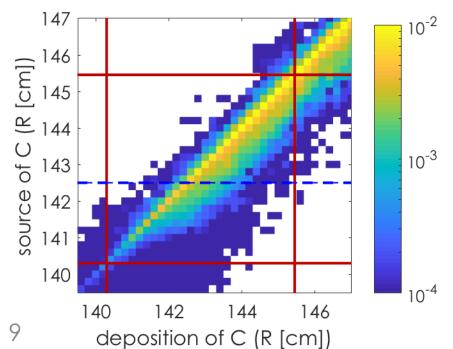
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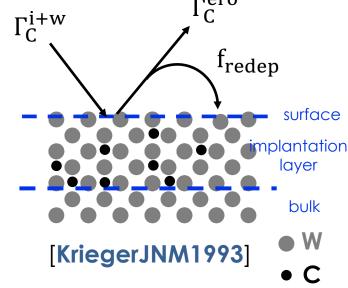


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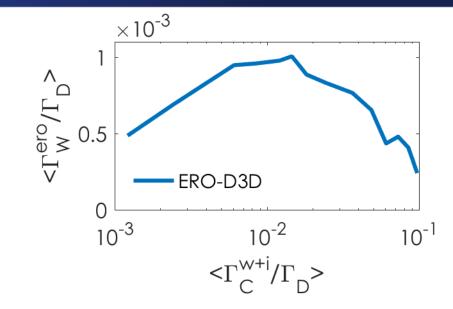


- Implantation of C in W and large C redeposition onto W induces large C flux on W (C "recycling")
- W gross erosion close to equilibrium at  $t\sim 1s$  , compatible with vs 5s DIII-D plasma
- Is this model actually robust against uncertainties?

#### W gross erosion weakly vary with C source due to the interplay between W sputtering by C and C implantation in W

- Weak dependency of W gross erosion on C source due to interplay between C implantation in W and W sputtering by C: model robust against uncertainties in C source!

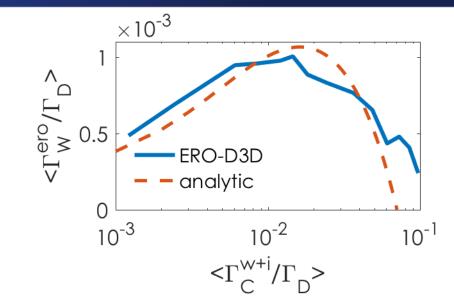
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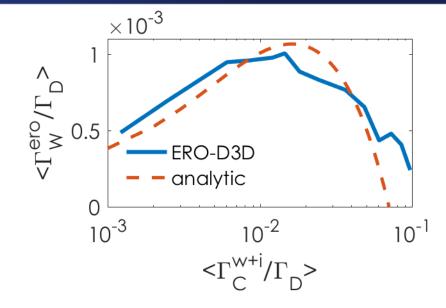
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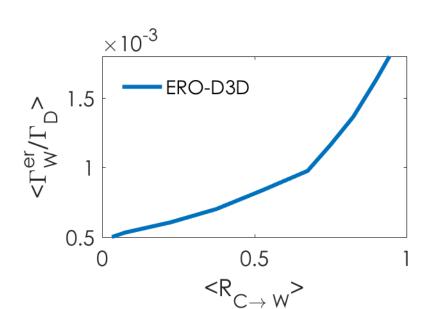
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$$<\frac{\Gamma_{\text{W}}^{\text{ero}}}{\Gamma_{\text{D}}}> \sim \left(1 - \frac{<\frac{\Gamma_{\text{C}}^{\text{W}+i}}{\Gamma_{\text{D}}}>}{<\frac{\Gamma_{\text{C}}^{\text{W}+i}}{\Gamma_{\text{D}}}>Y_{\text{C}\rightarrow\text{C}} + Y_{\text{D}\rightarrow\text{C}}(1 - f_{\text{redep}})}\right) \times \left(Y_{\text{D}\rightarrow\text{W}} + \frac{Y_{\text{C}\rightarrow\text{C}}}{1 - f_{\text{redep}}} < \frac{\Gamma_{\text{C}}^{\text{W}+i}}{\Gamma_{\text{D}}}>\right) \qquad 0$$



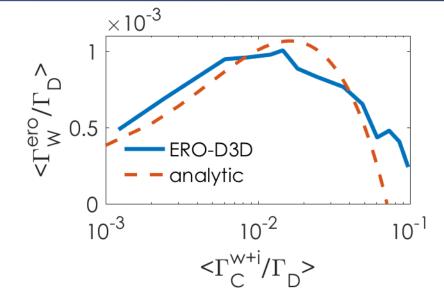
- C reflection on W strongly enhances W erosion:
  - Reflection of C on W = sputtering of W + instantaneous re-erosion
  - $R_{C \to W} \sim 0.7 0.8 > Y_{D \to C}$ ,  $Y_{C \to C}$



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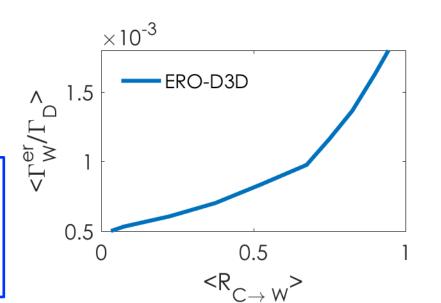
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W gross erosion induced by large C flux on W resulting from interplay between ExB drifts, C implantation/redeposition & reflection on W



### Modeling of C and W erosion/redeposition in DIII-D divertor

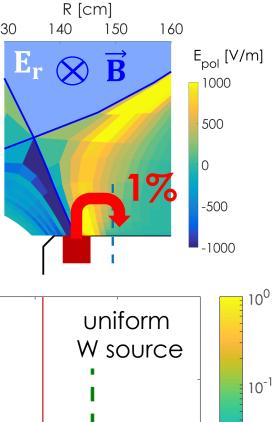
#### Introduction

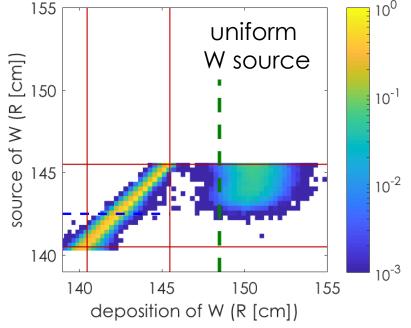
- Why modeling W net erosion is challenging?
- Measurement of W gross erosion and outboard deposition in DIII-D lower divertor with a toroidally symmetric W source

- Modeling and analysis of W gross erosion mechanism
  - W sputtering results from synergetic effects between impurity erosion, implantation, redeposition and transport processes
- Modeling and analysis of outboard W deposition mechanism
  - W net erosion may be inferred from W deposition measurements

## Outboard W deposition due to interplay between poloidal and radial ExB drifts and may be used to quantify W net erosion

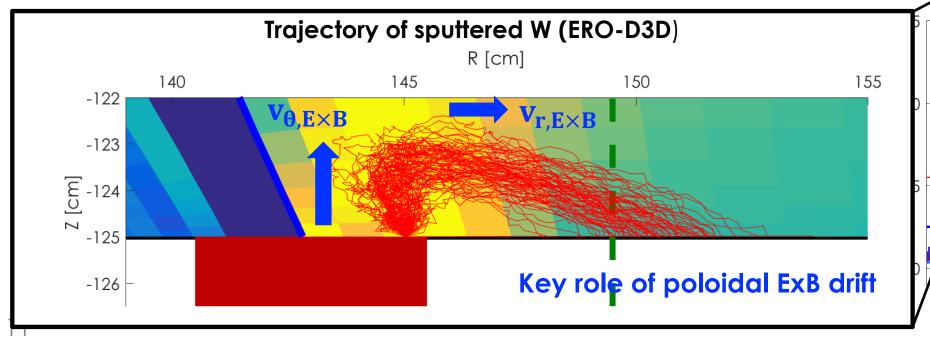
- Experiment: localized W deposition  $\Gamma_W^{dep}{\sim}0.01{\times}\Gamma_W^{ero}$  at 3.5cm from W outer edge
- Outboard W deposition qualitatively reproduced with ERO-D3D

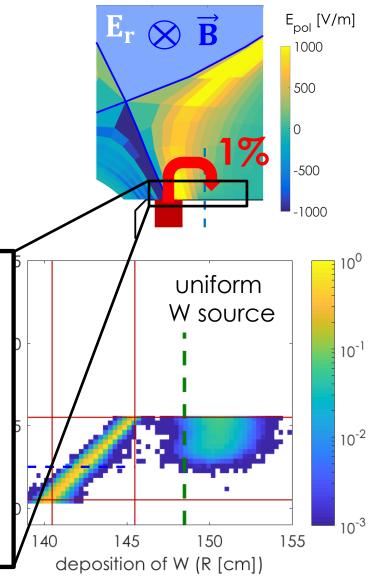




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  - Most of W not redeposited locally are deposited outboard



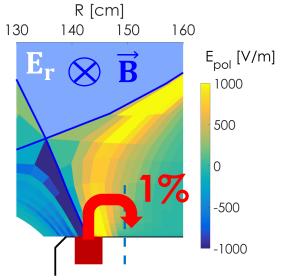


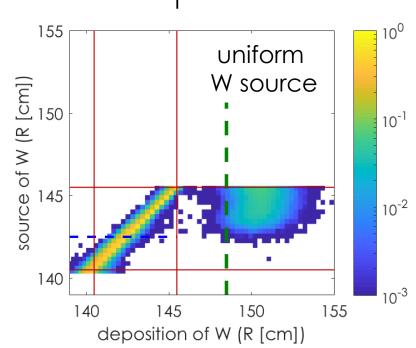
R [cm]

160

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  - Radial W migration due interplay between <u>outward</u> radial ExB drift and <u>upward</u> poloidal ExB drift (balance friction with D)
  - Most of W not redeposited locally are deposited outboard
- Measurement of W outboard deposition may help to quantify W net erosion...
  - But accurate quantitative modeling difficult due to uncertainties in plasma conditions (e.g. drifts near targets) and W transport (e.g. prompt deposition)





# Modeling of C and W erosion/redeposition in DIII-D divertor: conclusions

 Modeling of W erosion by low-Z impurities and W transport in divertor must include various physical mechanisms (mixed-material effects, ExB drifts, "global" source of low-Z impurity, reflection) and their synergetic effects to provide full consistency with plasma background conditions

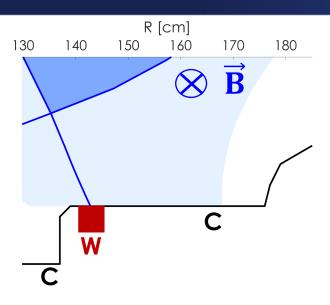
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- Modeling of W erosion by low-Z impurities and W transport in divertor must include various physical mechanisms (mixed-material effects, ExB drifts, "global" source of low-Z impurity, reflection) and their synergetic effects to provide full consistency with plasma background conditions
- Reduced model of material erosion with mixed-material (C and W described with the homogenous mixed-material model) sufficiently accurate to model material erosion:
- → It might be very beneficial to implement both reduced models (e.g. HMM) and advanced models of material erosion (e.g. SDTRIM.SP) in impurity transport code (e.g. GITR)
- Ideal framework to do numerical validations of GITR (e.g. against ERO and DIVIMP)
  and apply GITR to model impurity transport in Tokamak experiments

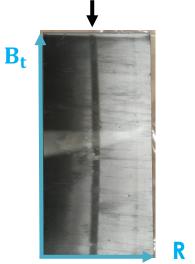
#### **Outline**

- Modeling of W ring experiments in DIII-D:
  - C and W erosion/redeposition in DIII-D divertor can be consistently modeled using a Monte-Carlo impurity transport code and sheath & material reduced models:
    - → Experimental and theoretical framework in DIII-D to validate and use impurity transport code in Tokamak conditions (GITR)
  - Accurate modeling of C deposition on W may however require a more detailed material model:
    - → Experimental framework in DIII-D to validate integrated models of surface evolution and roughness, material erosion and impurity transport
- Modeling W redeposition with ion-gyro sheath:
  - reduced model vs PIC model?
    - → Example of experimental framework in DIII-D to benchmark PIC simulations with ITER relevant physics

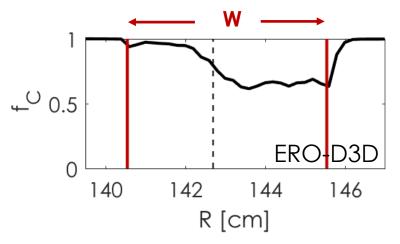
#### Carbon deposition observed on W at the separatrix location



C deposition strip on W near the strike point



- Net deposition of C on W ring at the separatrix location
- Net deposition of C roughly predicted with ERO-D3D in the private flux region but not at the separatrix:



- Homogenous mixed-material model cannot provide accurate modeling of C deposition on W [DrostePPCF2010]
- Surface roughness not included in the HMM, but may strongly affect C deposition on W [KreterPPCF2008]
- Modeling of C deposition on W observed during the metal ring campaign in DIII-D might be an good exercise to demonstrate the use of coupled models developed within the PSI-PsiDAC project (here Fractal-TriDyn+GITR)

#### **Outline**

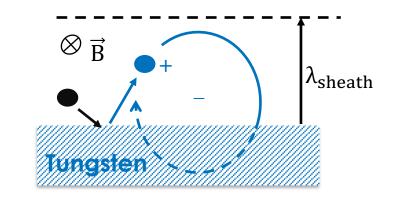
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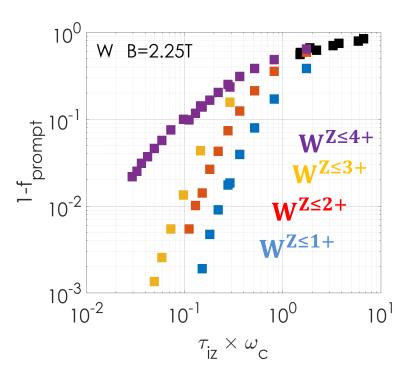
### Large W prompt deposition due to fast ionization of W within the ion-gyro sheath

- Fast ionization of W ( $\tau_{iz} \omega_c \ll 1$ ):
  - Large W prompt redeposition & W ionization within the sheath
- Sheath  $\approx$  ion gyro-sheath at grazing magnetic field incidence [RyutovCPP1996] ( $\lambda_{sheath} \sim \rho_i$ )
- Recent kinetic simulations [CoulettePPCF2016,StangebyNF2012] show  $\lambda_{sheath} \approx 5\rho_i$  ... but at B=10T
- Large effects of electron density decay in the sheath on W ionization and prompt redeposition (see e.g. [DingNF2016])

$$n_e^{sheath}(\hat{z}) = n_0^{plasma} e^{\hat{\phi}(\hat{z})}$$

- W prompt redeposition mainly governed by multiple ionizations of W in sheath
- Critical uncertainties for W prompt redeposition: ionization rates of W. "First-principle" model needed for W<sup>0+,1+,2+,3+,4+,5+</sup> ionization (see e.g. [SmythPRA18])

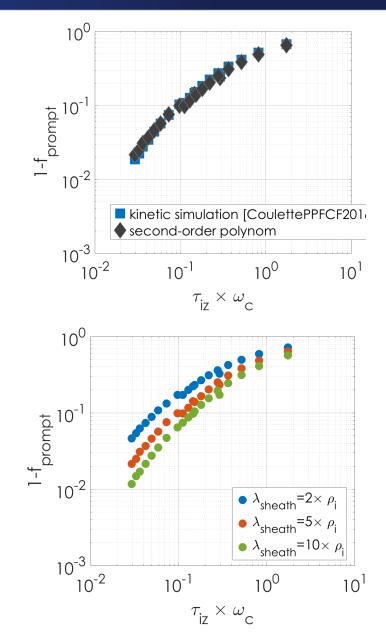




## W prompt deposition governed by W ionization rates and sheath scale length and can be described using a sheath reduced model

- Weak dependency of W prompt deposition on exact potential profile in the sheath:
  - $f_{iz}(\hat{z}) \sim \int_0^{\hat{v}_c} e^{-\frac{\int_0^{\hat{z}} e^{\hat{\phi}(\tilde{z})} d\tilde{z}}{\overline{\tau}_{iz} \hat{v}_z}} f(\hat{v}_z) d\hat{v}_z$
  - Allow complete analytical solution for W trajectory in the sheath(convenient for code validation with auto-adjusted timestep in the sheath)

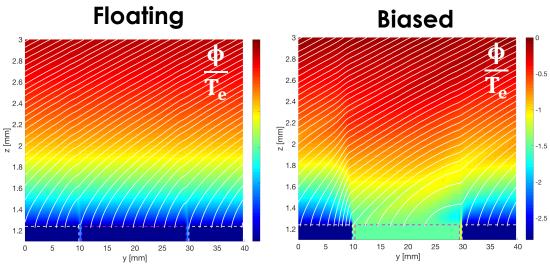
- Sheath length scale (but not the shape of the potential profile) has non-negligible effects on W prompt redeposition
- Can sheath length scale be well estimated in Tokamak divertor?
- If yes, PIC model of the sheath is not always necessary and reduced sheath model might be sufficient, e.g. for simple geometry and steady plasma conditions (≠ ELMs)



### But structure of the sheath may be more complex when considering real PFC geometry, e.g. near W tile edges in ITER W divertor ...

- PFC may exhibit complex geometrical features, e.g. W tile castellation and gap in ITER W divertor, which may strongly affect sheath and plasma conditions, and resulting PMI - erosion, melting,...
  - See for example R. Dejarnac talk<sup>1</sup> at PSI
- DiMES biasing experiments performed and modeled by R. Ding<sup>2</sup> at DIII-D exhibit similar geometrical effects on plasma:
  - Modification of the sheath due to gap between biased probe and DiMES head
  - Modification of the sheath due to biasing
  - Modeling of sheath and erosion with PIC and erosion/redeposition codes (here SPICE2/ERO)
- DiMES biasing experiments in DIII-D may provide an excellent framework to benchmark integrated PIC/impurity simulations in realistic Tokamak conditions with ITER relevant PMI physics





R. Dejarnac, Physics of toroidal gap heat loading on castellated plasma-facing components, PSI 2018
 R. Ding, Model validation on DIII-D experiments towards understanding of high-Z material erosion and migration in a mixed materials environment, PSI 2018

#### Conclusions

- Experimental and theoretical framework in DIII-D to:
  - validate and use impurity transport code in Tokamak conditions (GITR)
  - to validate integrated models of surface evolution and roughness, material erosion and impurity transport (Fractal-TriDyn+GITR)
  - to benchmark PIC simulations with ITER relevant physics (hPIC+GITR)

 Well diagnosed and controlled plasma conditions and versality of PFC material in DIII-D divertor provide an ideal benchmark to demonstrate the use of integrated/coupled complex PMI models developed in the PSI-PsiDAC project to analyze and model PMI physics in Tokamak experiments